Appendix F.

Glossary of Hanford Terminology
March 1996

This is a glossary of Hanford terminology that has been compiled to aid in definition of Hanford tank "jargon". These definitions have come from so many different sources that it is difficult to name them all. A lot of these terms have come from Anderson-91, Jungfleisch-84, and from Strode-93. Where there have been conflicting uses of the same term, it is indicated, and where there is uncertainty as to an exact meaning, a "??" appears to indicate that uncertainty.

If you have any corrections/additions/deletions to this glossary, please send them to: Stephen F. Agnew, M/S J586 Los Alamos National Laboratory, Los Alamos, New Mexico 87545, or fax to 505-667-0851.

ACL	Air Circulator lines (term located WHC-SD-WM-ER-204, Rev.0)
Active	Currently operating or scheduled for further operation
Active Drywell	Drywell in which radiation readings of greater than 50 counts/second are detected. To be considered "active", these readings must be consistent as to depth and radiation level for repeated readings.
Active Tank	A tank that contains more than 33,000 gal. of waste and/or is still involved in waste management operations.
ADD	Add primary waste from process.
ADJ	Adjustment to waste amount. See also CORR, COOL, and LEAK.
AEC	Atomic Energy Commission. See also ERDA, and DOE
AFPC	High total beta activity in the evaporator process condensate
AG	Above Grade (term located WHC-SD-WM-ER-204, Rev.0)
AGE	Aging Waste. See also AGING, AGING WASTE, HAW, IWW, NCAW, NFAW, NHAW, NRAW, PAW, PFM, and P83-88.
AGING	Aging Waste. See also AGE, AGING WASTE, HAW, IWW, NCAW, NFAW, NHAW, NRAW, PAW, PFM, and P83-88.
AGING WASTE	High level, first cycle solvent extraction waste from the PUREX plant See also AGE, AGING, HAW, IWW, NCAW, NFAW, NHAW, NRAW, PAW, PFM, and P83-88.
AIR LIFT CIRCULATOR	The air lift circulators are installed in aging tanks to promote mixing of the supernate. By maintaining motion within the body of the liquid, the circulators minimize superheat buildup and, consequently, minimize burping.
AL	Analytical Laboratories
ALARA	As Low As Reasonably Achievable
ALE	Fitzner-Eberhardt Arid Land Ecology Reserve
ANCHAR	Analysis of characteristic waste deriving waste compositions from analytical information.
ANL	Argonne National Laboratory
ANNULUS	The annulus is the space between the inner and outer shells on DSTs. Drain channels in the insulating and/or supporting concrete carry any leakage to the annulus space where conductivity probes are installed. (term located Tank and Surveillance and Waste Status Summary Report)
ANSI	American National Standard Institute
APC	Alpha proportional counting
A Plant	Where PUREX process ran from Jan. 1952 - Jun. 1972, then was in standby and ran again from Nov. 1983 - 1991, and is now shutdown). See also PUREX-Plant, CARB, CWP, and OWW
APM	Ammonium Phosphomolybdate (term located WHC-EP-0791)
AQUELLW	Aqueous liquids (term located WHC-EP-0791)
AR	“Washed” P sludge. Also used to derive SRR. See also SRR.

ARM	Area Radiation Monitor
AR Vault	PSL (PUREX sludge) was sluiced from A - and AX-Farms and placed here for caustic wash to remove Cesium and acid dissolution for feed to B Plant. AR-002 (or TK-002) was slurry receiver in AR-Vault. Solids are then transferred to TK-004, acidified, and the PAS (PUREX Acidified Sludge) transferred to TK-003. Any solids left in TK-004 following acid dissolution are caustic digested and transferred to back TK-002 for the next cycle.
ASF	Ammonia Scrubber Feed
ASME	American Society of Mechanical Engineers
Assumed Leaker	The integrity classification of a waste storage tank for which surveillance data indicate a loss of liquid attributed to a breach of tank integrity.
Assumed Leaking Tank	In 1984, the criteria designations of “suspect leaker”, “questionable integrity”, “confirmed leaker”, “declared leaker”, “borderline”, and “dormant” were merged into one category now reported as “assumed leaker”.
Assumed Re-Leaker	A designation that exists after a tank has been declared an Òassumed leakerÓ and then the surveillance data indicate a new loss of liquid attributed to a breach of integrity.
ASTM	American Society for Testing and Materials
AW	NEUTRALIZED CURRENT ACID WASTE
AWC	Aging Waste Condensate
A1SltCk	Salt cake waste generated from the 242-A Evaporator-Crystallizer from 1977 until 1980.
A2SltSlry	Salt Slurry waste generated from the 242-A Evaporator-Crystallizer from 1981 until 1994.
B86ON	DILUTE, NON-COMPLEXED WASTE FROM B PLANT CELL DRAINAGE
B	B Plant HLW. Also identifies waste returned to tanks from Sr recovery. Also used as destination, B Plant, for Cs/Sr recovery. $BiPO_4$ ran in B PLANT from Apr. 1945 to Oct. 1952, while Cs/Sr recovery from tank farms ran from 1967 to 1976, and Cs/Sr recovery from NCAW and CAW ran from 1967-72, and then from 1983-91. B Plant's mission from '67 was to take the acid stream from PUREX through Cesium and Strontium recovery operations.
BARCT	Best Available Radionuclide Control Technology
BAT/AKART	Best Available Technology/All Known And Relevant Technology
BC	TRU SOLIDS FROM B PLANT PROCESSING OF CC
BCD	Binary Code Decimal
BEMR	Baseline Environmental Management Report
BF	Breather Filter (term located WHC-SD-WM-ER-204, Rev.0)
BFSH	B Plant Flush
BG	Below Grade (term located WHC-SD-WM-ER-204, Rev.0)
BHI	Bechtel Hanford Inc.
BiPO4	Bismuth Phosphate Process. First precipitation process used at the Hanford Site for separating plutonium from the irradiated uranium fuels. This process was replaced by REDOX and PUREX processes to gain the advantages of separation and recovery of the uranium and plutonium fission products in B-222 and U-222, 1944-56. Left U in waste. See also MW, 1C, and 2C.
BIPP	B Plant Immobilization Pilot Plant
BIX	B Plant Ion Exchange
BIXBN	??
BIXRI	??
BL	B Plant Low Level. From '68-'76 added to AX-103, BX-101, B-101, and C-106. Wash(?) waste after concentration in cell 23 (i.e. low solids).
BLEB	B Plant Low level Evaporator Bottoms.
BLIX	B Plant Low Level Ion Exchange?

BLIXB	B Plant Low Level Ion Exchange bottoms?
BN	??
BNW	Battelle Northwest Laboratory Waste
Boiling Waste	Waste containing sufficient radioactive decay heat to self-boil.
Bottoms Receivers	Tank designated for receiving evaporator bottoms.
Bottom Referenced Tank	Either a dished bottom tank or a flat bottom tank where the zero point for liquid-level gages is the lowest elevation in the tank.
BP	TRU SOLIDS FROM B PLANT PROCESSING OF PFP
BPC	Beta proportional counting
BP/CPLX83-88	SSR, CSR, B, BL all in AY-101
BP/NCPLX83-88	now in AY-101
BPDCC	DILUTE, COMPLEXED WASTE FROM B PLANT CESIUM PROCESSING. See also CSR and BPDCC.
BPDCS	DILUTE, COMPLEXED WASTE FROM B PLANT STRONTIUM PROCESSING
BPDCV	DILUTE, COMPLEXED WASTE FROM B PLANT VESSEL CLEAN-OUT
BPFPS	B PLANT HIGH TRU SOLIDS FROM RETRIEVED PFP SOLIDS
B Plant	One of the three original Bismuth-Phosphate processing facilities. Later converted to waste fractional plant. B Plant used for $BiPO_4$ 1944-52, then for FP recovery. See also 222-B and TK.
BPLCS	DILUTE, NON-COMPLEXED WASTE FROM B PLANT STRONTIUM PROCESSING
BPLDC	DILUTE, COMPLEXED WASTE FROM B PLANT CESIUM PROCESSING
BPLDN	DILUTE, NON-COMPLEXED WASTE FROM B PLANT CESIUM PROCESSING
BR	TRU SOLIDS FROM B PLANT PROCESSING - NCRW
BS	B PLANT PRETREATED SOLIDS
B SLTCK	Salt cake waste generated from the 242-B Evaporator from 1951 until 1955.
BUMPING, TANK BUMP	A tank bump occurs when solids overheat in the lower portion of the tank. The hot solids are mixed with the cooler fluid either by operation of the airlift circulators (ACLs) or by natural means. The hot solids rapidly transfer heat to the liquid, some of which quickly vaporizes. The sudden pressurization caused by vapor generation is called a "bump".
Burial Ground (garden)	A land area specifically designated to receive packaged contaminated wastes and equipment for burial. Rated volume at the time of construction.
BVCLN	DILUTE, NON-COMPLEXED WASTE FROM B PLANT VESSEL CLEAN-OUT
BWIA	B Plant Waste Immobilization Annex. See also B Plant
BWIP	Basalt Waste Isolation Project.
BY SLTCK	Salt cake waste generated from in-tank solidification units 1 and 2 between 1965 and 1974.
Caisson	An underground structure used to store high-level waste; typical designs include corrugated metal or concrete cylinders, 55-gal. drums welded end-to-end, and vertical steel pipes below grade.
Calcine	To heat a substance to a high temperature, but below its melting point, causing loss of volatile constituents such as moisture; refers also to the material produced by this process.
CAM	Continuous Air Monitor
CARB	CARBONATED WASTEæsame as OWW. See also A Plant, PUREX Plant, CWP, and OWW.
CAS	Cascade, this process filled three or more tanks with one pump by using overflow lines. Normal use was with a sequence of tanks numbers 101, 102, 103, or 110, 111, 112. See also SET and END.

Cascade	Eleven of the Single-Shell Tank Farms (all except the AX-Tank Farm), were equipped w/ overflow lines between tanks. The tanks were connected in series and were placed at different elevations creating a down hill gradient for liquids to flow from one tank to another. See also CAS, SET, and END.
CASS	Computer Automated Surveillance System (AY and AZ Farm)
Catch Tank	Small-capacity single-wall tank, primarily associated with diversion boxes and diverter stations. The tanks collect liquid from diversion boxes, diverter stations, catch stations, and other facilities.
CAW	Current Acid Waste—this is PUREX acid waste, also called HAW or IWW. See also HAW, IWW, and PAW.
CB	??
CBUSTL	Combustible Solids and Liquids
CC	COMPLEXANT CONCENTRATE. Term refers to concentrates of solutions that have TOC's greater than 10 g/L. Usually associated with EDTA and HEDTA salts. See also CCPL, CCPLX, and CPLX.
CCGL	B PLANT HIGH TRU SOLIDS FROM RETRIEVED COMPLEXED CONCENTRATE
CCGR	DILUTE, NON-COMPLEXED WASTE FROM RETRIEVED COMPLEXED CONCENTRATE
CCPL	COMPLEXANT CONCENTRATE. See also CC, CCPLX, and CPLX
CCPLX	Complexant Concentrate. See also CC, CCPL, and CPLX
CCW	Complex Concentrated Waste
CCW	Concentrated Customer Waste
CCW	Counter-Clockwise ref. (LA-UR-92-3196)
CD	??
CDE	Committed Effective Dose Equivalent
CDF	TRAC Composition Data File or Transaction Flag Key—unit volume assumed to make stream active.
CE	Evaporator Concentrate
CE	Crown Ether
Cell 23	Waste from Cell 23 at B Plant. Cell 23 contained an evaporator and was used not only during B Plant operations, but to reduce tanked waste as well.
CEM	Cement added to BY-106 in 1977, see also CON.
CERCLA	Comprehensive Environmental Response, Compensation and Liability Act.
CF	Cesium Feed
CFR	*Code of Federal Regulations*
CHP	Cascade Heel Pit
C Layer	Convective Layer
CLEAN 31	CLEAN Option HLW stream
CLELLW	CLEAN Option LLW stream
CLU	Chemical Laboratory Unit
CMPO	N-diisobutylcarbmoylmethylphosphine oxide
CON	Cement added to BY-105 in 1977, see also CEM. Also designated concentrated waste in SX-103 (1965-66), SX-107 (1965), SX-108 (1965), and SX-110 (1965).
COND	CONDENSATE. See also EVAP, AND EB.
COND	Condition
Conductivity Probe	Measures surface level of conductive liquid (or waste) by detecting electrical conductivity between probe tip and liquid/waste surface as it is lowered into contact.
Confirmed or Declared Leaker	The designation of any underground waste storage tank where the data is considered sufficient to support a conclusion with 95 percent confidence that the tank has leaked.

COOL	Change in waste volume due to cooling. See also ADJ, COOL, CORR, and LEAK.
CORR	Correction to tank waste level. See also ADJ, COOL, and LEAK.
CP	Condenser Pit
CP	CONCENTRATED PHOSPHATE WASTE (FROM 100 N-REACTOR DECONTAMINATION). See also N.
C Plant	Strontium Semiworks. Called C Plant or Hot Semiworks earlier, was pilot for both REDOX and PUREX, Jul. 1952 to Jul. 1956. Then reconfigured for Strontium Recovery Pilot Plant from July 1960 to July 1967. See also 222-C, SSW, and HS.
CPLX	Complexed waste. See also CC, CCPLX, and CCPL.
CPP	Cascade Pump Pit
CPW	Concentrated Phosphate Waste. Waste originating from the decontamination of 100-N Area reactor. concentration of this waste produces concentrated phosphate waste.
CRIB	Ground site for low level supernatants (from tanks) or condensates (from evaporators). NW (T-105 - T-107, T-018, T-021 - T-023, T-025, T-026, T-032, TY-CRIB, TY-1) and NE (B-##, S-##, T-##, A-008, A-024, B-007, B-008, B-014, B-016, B-018, B-035, B-037, B-040, B-042, and B-049).
CRUST	A hard surface layer that has formed in many waste tanks containing concentrated solutions.
CR Vault	Facility located adjacent to C Farm, used for scavenging campaign following Uranium recovery, 1952-58. Ferrocyanide was added to tank supernatants in CR-Vault, and then the slurry was returned to C Farm for settling, forming in-farm sediments.
CRW	Cladding Removal Waste
CSFD	Cesium Feed
CSIX	Cesium ion Exchange
CSKW	??
CSP	Cascade Sluice Pit
CSR	Tank supernatant was sent to B Plant for Cesium recovery using C-105 as a staging tank. From 1967-76, 21,724 kgal was sent to and 26,290 kgal returned from B Plant. See also IX, and BPDCC.
CSS	Concentrated supernatant solids
CST	Caustic Solution, 0.01 M NaOH.
CSWLE	COMPLEXED SALT WELL LIQUID EAST AREA
CSWLW	COMPLEXED SALT WELL LIQUID WEST AREA
CTW	Caustic waste for makeup
CUWP	Chemicals Used and Waste Volume Produced
CVAA	Cold vapor atomic absorption (Waste)
CVR	Metal Cover Plate
CVS	Compostion Variability Study
CW	Cladding Waste, included with 2C from 1945-50, and with 1C from 1951-56.
CW-Al	Aluminum cladding waste
CWHT	Concentrated Waste Holding Tank
CWP	Cladding Waste PUREX. See also A Plant, PUREX Plant, and OWW.
CWP2	Cladding waste. PUREX 2?
CWR	Cladding Waste-REDOX. See also REDOX and R.
CWR1	REDOX cladding waste from 1952 to 1960.
CWR2	REDOX cladding waste from 1961 to 1967.
CWZr1	Cladding waste from PUREX 1966-70 that used Zirflex process on Zircaloy clad fuel elements. See also PD and NCRW.
CWZr2	Coating waste (REDOX), zirconium cladding

CWP/Zr83-88	now called PD or NCRW
CX70	DILUTE, COMPLEXED (MIXTURE) HOT SEMI-WORKS TRU SOLIDS
D	Dilute
DACS	Data Acquisition Control System
DAS	Data Acquisition System
DBA	Design Basis Accident
DBP	Dibutyl Phosphate
DBPW	Dilute “B” Plant Waste
DC	DILUTE COMPLEXED. Waste characterized by a high content of organic carbon including organic complexants: ethylenediaminetetra-acetic acid (EDTA), citric acid, hydroxethylenediaminetriacetic acid (HEDTA), and iminodiacetate (IDA) being the major complexants used. Main sources of dilute complexed waste in the double-shell tanks system are salt well liquid inventory. See also, EDTA, HEDTS, and IDA
D & D	Decontamination and Decommissioning
DCG	Derived Concentration Guide
DCH 18-Cr-6	Dicyclohexano 18-Crown-6 Ether
DCS	Dilute Caustic Solution
DCW	Dilute Complexed Waste
DDSSF	Dilute Double Shell Slurry Feed
DDT	Deflagration to Detonation Transition
DDWSF	Dilute Double-Shell Slurry Feed. Product from run 86-1. See also DSS, and DSSF.
DE	Diatomaceous Earth added to BX-102 (1971), SX-113 (1972), TX-116 (1970), TX-117 (1970), TY-106 (1972) U-104 (1972).
DEF	??
DF	Decontamination Factor (term located WHC-EP-0791)
DIL	Dilute Feed for Evaporator input. Interstitial liquid that is not held in place by capillary forces, and will therefore migrate or move by gravity. See also DILFD
DILFD	Dilute Feed. See also DIL.
DISS	Dissolver
Ditch	A linearly oriented excavation often used for the temporary diversion or disposal of process waste streams.
Diversion Box	A below-grade concrete enclosure containing the remotely maintained jumpers and spare nozzles for diversion of waste solution to storage tank farms.
DN	DILUTE NON-COMPLEXED WASTE (DN) (i.e. contains no complexants) defined as waste with TOC <1wt% (10 g/L). See also DN/PD, DN/PT, PFP, PRF, TRU Solids, TRU, Z, and 224
DNCPW	Dilute Noncomplexed Waste
DN/PD	Dilute Non-Complexed Waste (DN) with P TRU solids. See also DN, DN/PT, P, PFP, PRF, PRF TRU Solids, TRU, Z, and 224..
DN/PT	Dilute Non-Complexed Waste (DN) with PFP TRU solids. See also DN, DN/PD, P, PFP, PRF, PRF TRU Solids, TRU, Z, and 224.
DNSFB	Defense Nuclear Facilities Safety Board
DoD	US Department of Defense
DOE	US Department of Energy. See also AEC and DOE.
DOE/RL	DOE/Richland (Field Office)
DOH	Washington Department of Health
DP	DILUTE PHOSPHATE WASTE
DP	Differential Pressure (term used LA-UR-92-3196 Rev 0)
DP	Distributor Pit (term used WHC-SD-WM-ER-204, Rev.0)
DPDS	Dilute PUREX Decladding Supernate

Drainable Interstitial Liquid	Liquid that is not held in place by capillary forces, and will therefore migrate or move by gravity. Drainable liquid remaining minus supernate. Drainable Interstitial Liquid is calculated based on the salt cake and sludge volumes, using average porosity values or actual data for each tank, when available.
Drainable Remaining Liquid	Supernate plus drainable interstitial.
DRCVR	Dilute Receiver Tank
DRYWELL	Vertical boreholes with 6-inch (internal diameter) carbon steel casings positioned radially around single-shell tanks. Periodic monitoring is done by gamma radiation or neutron sensors to obtain scan profiles of radiation or moisture in the soil as a function of well depth, which could be indicative of tank leakage. These wells range between 50 and 250 feet in depth, and are monitored between the range of 50 to 150 feet. The wells are sealed when not in use. The wells are called drywells because they do not penetrate to the water table and are therefore usually “dry”.
Drywell (in tank)	A sealed casing within a tank that is attached to a riser and used for access of a gamma or neutron detector, or an acoustical probe to determine the level of interstitial liquid.
DSS	DOUBLE-SHELL SLURRY (from EOFY 77 inventory?). This waste is a concentrate of DSSF, but with a TOC<10g/L (<1wt% TOC is NC). Waste that exceeds the sodium aluminate saturation boundary in the evaporator without exceeding receiver tank composition limits. DSS is considered a solid. See also DDWSF and DSSF
DSSF	DOUBLE-SHELL SLURRY FEED. Waste concentrated just before reaching the Sodium Aluminate saturation boundary in the evaporator without exceeding receiver tank composition limits. This form is not as concentrated as DSS. See also DSS and DDWSF.
DST	Double Shell Tank. The newer one million gallon underground waste storage tanks consisting of a concrete shell and two concentric carbon steel liners with an annular space between the liners.
DTPA	diethylene-triamine-penta-acetic acid (term located WHC-EP-0791)
DUMM, DUMMY	Dummy Waste.
DW	Decontamination Waste
DWBIX	DECONTAMINATION WASTE AND B PLANT ION EXCHANGE
DWPF	Defense Waste Processing Facility
DWVD	Defense Waste Vitrification Demonstration
E	Emergency
E-Stop	Emergency stop
EAC	Energy Absorption Capacity
EB	Evaporator Bottoms. See also COND and EVAP.
Ecology	Washington State Department of Ecology
EDE	Effective Dose Equivalent
EDTA	Ethylenediaminetetraacetic acid (term located WHC-EP-0791). See also, DC, HEDTA, and IDA
EF	Evaporator Feed
EFD	Evaporator Feed Dilute
EGR	Episodic Gas Release (term located WHC-EP-0702, Rev 0)
EIS	Environmental Impact Statement
ELEVATION	Surveyed at riser flange (term used SD-RE-TI-053 Rev. 8)
END	Disconnect Cascaded Tanks. See also CAS, and SET.
EP	Enclosure Pit (term used WHC-SD-WM-ER-204, Rev.0)
ERA	Expedited Response Action
ERDF	Environmental Restoration Disposal Facility
EPRI	Electric Power Research Institute

ERPG	Emergency Response Planning Guideline
ERDA	Energy Research and Development Administration. See also AEC, and DOE.
ES&H	Environment, Safety, and Health
ESPIP	Efficient Separations and Process Integrated Program (term used WHC-EP-0791)
ETF	Effluent Treatment Facility
EV	Evaporation
EV	Evaporation Entry
EVAP	EVAPORATOR LOSSES
EVAP	Evaporator connected to tank. See also COND and EB.
EVAP	Evaporator Feed (post 1976)
EVAPF	DILUTE, NON-COMPLEXED WASTE FROM EVAPORATOR PAD FLUSH
EVAP Feed	Any waste liquid that can be concentrated to form salt cake; e.g., aged waste, low heat waste, dilute interstitial liquor, and other radioactive waste solutions.
Evap Feed Dil	Evaporator Feed Dilute. See also EFD
EVFD	Evaporator Feed Tank
EVS	Partial neutralization in 242-S Evaporator.
EVT	HEDTA destruction in 242-B or 242-T evaporators.
Evaporator Crystallizer	242-A and 242-S waste concentration facilities that operate at a reduced pressure (vacuum) and are capable of producing a slurry containing about 30 volume percent solids at a specific gravity of greater than 1.6.
Evaporator Feed	Any waste liquid that can be concentrated to form salt cake; e.g., low heat waste, dilute interstitial liquor, aged waste, and other radioactive waste solutions.
F	Food Instrument Company (FIC) Automatic Surface Level Gauge (term used Tank and Surveillance and Waste Status Summary Report)
FAILED	Thermocouples with either open circuits or loop resistance. (term used WHC-SD-WM-TI-553, Rev.0)
F/B	flange with bale (term used WHC-SD-WM-ER-204, Rev.0)
FCT	flux-corrected transport
FD	Feed Dilute
FDC	functional design criteria
FeCN	Ferrocyanide wastes created during a scavenging campaign in 1953-57. See also SCAV, P00, T00, PFeCN1, PFeCN2, and TFeCN
FFTF	Fast Flux Test Facility
FIC gauge	A Food Instrument Corporation Automatic Liquid Level Gauge based on a conductivity probe. At Hanford they are electrically connected to a computer for data transmission, analysis, and reporting. Local readings may also be obtained from a dial. (term located Tank and Surveillance and Waste Status Summary Report)
FIRST AND SECOND CYCLE DECONTAMINATION WASTES	Waste contained 10 percent of the original fission product activity and 2 percent of the product. By-product cake solution was mixed with product waste and neutralized with 50 percent caustic. This waste contained a mixture of suspended solids, hydroxides, carbonate and phosphate, scavenger metals, and chromium, iron and sodium, silicofluoride. See also 1C and 2C.
F/L	Flange with lead
FLSH	Flush water.
FM	Flow meter (term located LA-UR-92-3196 Revised)
FM-Approved	Factory Mutual-Approved (term located LA-UR-92-3196 Revised)
FP	Fission Product Waste. Cs and Sr recovery began in 222-B in 1967. Cs was removed from PUREX SU (PAW) and Sr from PUREX SL (PAS), and both from Acidic Waste.
FSPLIT	Separates or slots the flow of one or more input streams into two or more output streams.

FTIR	Fourier Transform Infrared (term located WHC-EP-0702, Rev 0)
FV	Field Verify
GA	Gain to Tank
GAS	SLURRY GROWTH AS A RESULT OF GAS GENERATION
GC	Gas Chromatograph (term located LA-UR-92-3196 Revised)
GEA	Gamma Energy Analyses (see SD-WM-PE-029 Rev. 0, 242-A Evap/Crystallizer FY 84-86 Campaign Run.
GIT	Georgia Institute of Technology (term located WHC-EP-0702, Rev 0)
GM Instrument	Instrument for detecting low-level beta and gamma radiation using a Geiger-Mueller tube.
GRD	Riser at Grade (term located WHC-SD-WM-ER-204, Rev.0)
GRE	Gas Release Event (term located WHC-EP-0702, Rev 0)
GROUP	A group of tanks where ITS averaged the supernatant phases. See also ITS.
GROUT	OUTFLOW TO THE GROUT FACILITY
GRTFD	Grout Feed Tank
GTCC	Greater than Class C (term from WHC-EP-0791)
GUNITE	A building material consisting of a mixture of cement, sand, and water that is sprayed onto a mold.
HAMMER	Hazardous Materials Management and Emergency Response Training Center
Hanford Coordinates	A set of offsets, in feet, from a reference point on the site. These are the units used to lay out these facilities. Conversion to latitude and longitude is possible.
Hard Pan	Term used to describe uranium carbonate phase that formed in solids from MW additions. Proved to be very difficult to sluice.
HASP	Health and Safety Plan
HAW	Aging waste from PUREX/PFM Processing NPR Nuclear Fuel. See also AGE, AGING, AGING WASTE, IWW, NCAW, NFAW, NHAW, NRAW, PAW, PFM, and P83-88.
HazOP	Hazards and Operability Study
HDRL	Hanford Defense Residual Liquid
HEAT	A tank level correction due to thermal expansion. See also CORR, COOL, and LEAK.
HEDL	Dilute sulfate waste. See also UNC.(see SD-WM-PE-029 Rev..0, 242-A Evap/Crystallizer FY 84-86 Campaign Run)
HEDTA	N-(2-hydroxyethyl)ethylenediamine tetra acetate
Heel	The waste that remains in a tank after the tank is emptied.
HEPA	High-Efficiency Particulate Air . A filter designed to achieve 99,995 percent minimum efficiency in the containment of radioactive particulates greater than 0.3 micrometer in size. (term located WHC-EP-0702, Rev 0)
HFW	Hanford Facility Wastes
HHI	Health Hazard Index (term from WHC-EP-0791)
HHW	High Heat Waste
HIC	High Integrity Container
HJ	Heel Jet (term from WHC-SD-WM-ER-204, Rev.0)
HLO	Hanford Laboratory Operations Waste
HLW	High-Level WasteÑgeneric for all Hanford Tank Wastes. Waste from the fuel reprocessing operations in separations plants.
HP	Heel Pit (term from WHC-SD-WM-ER-204, Rev.0)
HMS	Hanford Meteorological Station
HMS/TRAC	Hydrogen Mixing Study Transient Reactor Analysis Code (term located LA-UR-92-3196 Revised)
HS	Hot Semiworks. A pilot facility that had a variety of operations. See also C Plant, and SSW.

HSA	Hanford Strategic Analysis (term located WHC-EP-0791)
HSRAM	Hanford Site Risk Assessment Methodology
HTCE	*Historical Tank Content Estimate*
HTWRS	Hanford Tank Waste Remediation System
HVAC	Heating, Ventilating, and Air Conditioning
HWVP	Hanford Waste Vitrification Plant.
HWVP	DILUTE, NON-COMPLEXED WASTE FROM THE VITRIFICATION PLANT (term From WHC-EP-0791)
I&S	Tank Isolated and Stabilized
IC	Synonym (misspelling?) for 1C-1st cycle decontamination waste-$BiPO_4$. See also MW, 2c, and $BiP0_4$.
ICE	Implicit Continuous Eulerian (term located LA-UR-92-3196 Revised)
ICEBC	?? (1st cycle evaporator bottoms concentrate??) See 1CEBC
ICF	Consolidated Incinerator Facility (term located WHC-EP-0791)
ICO	DILUTE NON-COMPLEXED WASTE FROM TERMINAL CLEANOUT.
IDA	Iminodiacetate. See also, DC, EDTA, and HEDTA.
IDEF	Integrated Computer-Aided Manufacturing (ICAM) Definition (Language) (term located WHC-EP-0791)
IDLH	Imminently (or immediately) Dangerous to life or health (term located LA-UR-92-3196 Revised)
Inactive Tank	A tank that has been removed from liquid-processing service, has been pumped to less than 33,000
IH	Instrument House (term from WHC-SD-WM-ER-204, Rev. 0)
II	Interim Isolated. The administrative designation reflecting the completion of the physical effort required to minimize the addition of liquids into an inactive storage tank, process vault, sump, catch tank, or diversion box. In June 1993, Interim Isolation was replaced by Intrusion Prevention. (term located Tank and Surveillance and Waste Status Summary Report)
ILL	Interstitial Liquid Level**.** Liquid that resides in the voids/interstices of the solids.
Inactive Tank	A tank that has been removed from liquid processing service, has been pumped to contain less that 33,000 gallons of waste, and is not yet or in the process of stabilization and interim isolation. This includes all tanks not in active or active-restricted categories. Also included are inactive spare tanks that would be used if an active tank failed.
INEL	Idaho National Engineering Laboratory (term located WHC-EP-0791)
In-Service Tank	The waste classification of a tank being used, or planned for use, for the storage of liquid (in excess of a minus supernatant liquid heel) in conjunction with production and/or waste processing. All Hanford double-shell tanks are in-service; none of the single-shell tanks are in-service.
INST	CHANGE IN TANK LEVEL DUE TO CHANGE IN INSTRUMENTATION.
Interim Isolation	An administrative designation reflecting the completion of the physical effort required to minimize the addition of liquids into an inactive storage tank, process vault, sump, catch tank, or diversion box. See Intrusion Prevention.
Interim Stabilization	A tank which contains less than 50,000 gallons of drainable interstitial liquid and has less than 5,000 gallons of supernatant. If the tank was jet pumped to achieve interim stabilization, then the jet pump flow must have been at or below 0.05 gallons per minute before interim stabilization is completed.
Intrusion	The unintended entry of any liquid into a waste storage tank.

Intrusion FIC	A mode of operating the FIC surface level monitoring equipment typically used when a waste surface is non-electrically conductive. The conductivity probe (plummet) is positioned a small distance above the waste surface. Should that gap be spanned by an intruding liquid, conductivity between the plummet and the waste surface would be established this triggers an alarm in the CASS system. Note that the intrusion FIC levels is not an actual measurement of the current waste surface.
Intrusion Mode FIC Setting	The FIC probe is positioned a short distance above the waste surface. If the surface level of the waste in the tank increases, thereby touching the probe tip, a pointive indication is received.
IP	Intrusion Prevention. This is an administrative designation reflecting the completion of the physical effort required to minimize the addition of liquid into an inactive storage tank, process vault, catch tank, sump, or diversion box. (term located Tank and Surveillance and Waste Status Summary Report) See also IP.
IP	Instrument House (term from WHC-SD-WM-ER-204, Rev.0)
IRAP	Integrated Risk Assessment Program
IS	Interim Stabilized. A tank which contains less than 50,000 gallons of drainable interstitial liquid and has less than 5,000 gallons of supernatant liquid. If the tank was jet pumped to achieve interim stabilization, then the jet pump flow must also have been at or below 0.05 gallons per minute before interim stabilization is completed.
ISO	Tank is Interim-Isolated
Isolation	The act of sealing a tank against liquid intrusion from credible sources and confining the atmosphere in the tank. Filtered airways are not sealed. The balance the pressure to the atmosphere, and in some cases provide cooling airflow.
ISV	In-situ Vitrification (term located WHC-EP-0791)
ITS	In-Tank Solidification-Program using steam evaporators inside of certain tanks on BY Farm. ITS#1 ran 1965-70 in BY-102 (a pilot demonstration was also run in BY-101) and ITS#2 ran 1968-74 in BY-112. During 1971-74, ITS#1 used as cooler instead of a heater. See also GROUP
IWW	INORGANIC WASH WASTE TO SSTÑsame as P or NCAW. Refers to HAW or PAW. See also AGE, AGING, AGING WASTE, HAW, NCAW, NFAW, NHAW, NRAW, PAW, PFM, and P83-88.
IX	Ion Exchange Waste. Assumed ion exchange (IX) removal efficiency for radionuclides (i.e., americium, strontium, cesium, and technetium). Ion Exchange identifies waste returned from Cs recovery. See also CSR, and BPDCC.
IXROW	??Ion-Exchange REDOX Organic Wash??
JEG	Joint Evaluation Group (term located LA-UR-92-3196 Revised)
JET PUMP	A modified commercially available low capacity jet pump used as a salt well pump.
KNUCKLE	Point where the side wall and the bottom curved surface of a tank meet.
KOP	Knowledge of Process uses process information to derive waste compositions based on some process driver.
L	Inactive/Leaker
LaF	Lanthanum Fluoride waste generated in Plutonium Finishing Plant Operation from 1945-??. See also 224, and 224-F.
LANCE	OUT FLOW DUE TO LANCING OF TANK
Lance/Lancing	A long steel pipe, usually 2-to-3 inches in diameter. The top is bent at a 90-degree angle, and contains a check valve, gate valve, and nose connection. The bottom end of the lance is tapered to a 1/2-inch diameter. Water enters the top of the lance, which is forced out the bottom at high pressure. This creates a passage way which may be used for equipment installation.
LANH	Heavy Lanthanides (term located WHC-EP-0791)
LANL	Los Alamos National Laboratory

LANL	Light Lanthanides (term located WHC-EP-0791)
LATA Consortium	Los Alamos Technical Associates; British Nuclear Fuels, LTD; Southwest Research Institutes; and TRW, Inc.
Lateral	Horizontal drywell positioned under single-shell waste storage tanks to detect radionuclides in the soil which indicate leakage. Lateral drywells are monitored by radiation detection probes. Laterals are 4-inch ID steel pipes located 8 to 10 feet below the tankÕs concrete base. There are three laterals per tank in A and SX Farms. There are no lateral drywells in any other farms.
LB	Lifting Bale. Riser top has plate flange with lifting bale - possible concrete plug under
LE	Lead Encasement (term From WHC-SD-WM-ER-204, Rev.0)
LEAK	Tank leak volume. See also ADJ, COOL, and CORR.
LEAK DETECTOR	Fixed liquid level sensor - tape with weight (term located SD-RE-TI-053 Rev. 8)
LEAK DETECTION PIT	Collection point for any leakage from AM Farm Tanks. The pits are equipped with radiation and liquid detection instruments.
LEL	Lower Explosive Limit (term located WHC-EP-0702, Rev 0)
LERF	Liquid Effluent Retention Facility.
LETF	LIQUID EFFLUENT TREATMENT FACILITY FROM N REACTOR.
Level Adjustment	Any update in the waste inventory (or tank level) in a tank. The adjustments usually result from surveillance observations or historical investigations.
Level History	A diagram that shows the history of the waste level and waste level changes in a tank. The diagram also includes other related data.
LFL	Lower Flammability Limit (term located WHC-EP-0702, Rev 0)
Liquid Level Best Engineering Judgment Line	During the initial filling of certain single-shell tanks, only the liquid level was reported. To adjust for the big increase in level height, which occurred when solids were added to the record, a sloped line was used to reflect solids volume between the initial fill and the time the solids data were recorded.
LIT	Automatic Liquid indicator Tape (term located SD-RE-TI-053 Rev. 8)
LLI	Manual Liquid Level Indicator (term located SD-RE-TI-053 Rev. 8)
LLR	liquid level reel (term located WHC-SD-WM-ER-204, Rev.0)
LLR	manual liquid level sensor - tape with weight (term located SD-RE-TI-053 Rev. 8)
LLW	low-level waste (term From WHC-EP-0791)
LO	Loss from tank. (term From WHC-SD-WM-ER-204, Rev.0)
LOW	Liquid Observation Well. Liquid observation wells are used for monitoring the interstitial liquid level (ILL) in single-shell waste storage tanks. The wells are constructed of fiberglass, or tefzel-reinforced epoxy-polyester resin. They extend to within 1 inch of the bottom of the tank steel liner. They are sealed at their bottom ends and have a nominal outside diameter of 3.4 inches. See also ADJ, COOL, and CORR.
LUNC	DILUTE, NON-COMPLEXED WASTE FROM UNC FUELS FABRICATION FACILITY
LW	Laboratory Waste
L222S	222S LAB DILUTE NON-COMPLEXED WASTE FROM S PLANT.
L3A4A	DILUTE NON-COMPLEXED LABORATORY WASTES FROM 300 AND 400 AREAS.
M	Manual Tape Surface Level Gauge (term located Tank and Surveillance and Waste Status Summary Report)
MAB	Maximum Allowable Burp (term located LA-UR-92-3196 Revised)
MAPs	Mitigation Action Plans
MARGINAL	Thermocouple with higher than normal (0.5 ohms to 20 ohms depending on length) loop resistance, higher than normal resistance in one lead to ground, or having some other abnormality, e.g. inconsistent resistance measurements. (term located WHC-SD-WM-TI-553, Rev.0)

MAWB	Maximum Allowable Window Burp (term located LA-UR-92-3196 Revised)
MAXSPD	Maximum Speed Parameters (term located LA-UR-92-3196 Revised)
MCC	Motor Control Center (term located LA-UR-92-3196 Revised)
MDW	Miscellaneous Dilute Waste
MEB	Maximum Expected Burp (term located LA-UR-92-3196 Revised)
MIE	Minimum Ignition Energy (term located WHC-EP-0702, Rev 0)
MIT	Multifunction Instrument Tree (term located WHC-SD-WM-TI-553, Rev 0)
MPR	Multiport Riser (term located LA-UR-92-3196 Revised)
MS	Mass Spectrometer (term located LA-UR-92-3196 Revised)
MW	Metal Waste from $BiPO_4$. 90% of FP, all of U, 1% of Pu . Waste from the extraction containing all the Uranium, approximately 90% of the original fission product activity, and approximately 1% of the Pu product. This waste was brought just to the neutral point with 50% caustic and then treated with and excess of sodium carbonate. This procedure yielded almost completely soluble waste at a minimum total volume. The exact composition of the carbonate compounds was not known but was assumed to be a Uranium Phosphate Carbonate mixture. See also 1C, and 2C.
MW	Maximum Window (term located LA-UR-92-3196 Revised)
MW1	Metal waste from $BiPO_4$, 1944 to 1951
MW2	Metal waste from $BiPO_4$, 1952 to 1956
MWB	Maximum Window Burp (term located LA-UR-92-3196 Revised)
MWF	Metal Waste Feed? Set to water in TRAC.
N	N-Reactor waste. See also CP.
N2	Nitrogen
NBAW	NEUTRALIZED B PLANT ACID WASTE
NCAW	LIQUID WASTE, HIGH CS, SR, AND TRU CONTENT. Neutralized Current Acid Waste primary HLW stream from PUREX process. See also AGE, AGING, AGING WASTE, HAW, IWW, NFAW, NHAW, NRAW, PAW, PFM, and P83-88.
NCBUSTS	Noncombustible Solids (term located WHC-EP-0791)
NC layer	Nonconvective Layer (term located LA-UR-92-3196 Revised)
NCPL	Non-Complexed Waste general term applied to all Hanford site liquids not identified as complexed. See also NCPLX and NCPLEX.
NCPLEX	Non-Complexed Waste. See also NCPL and NCPLX.
NCPLX	Non-Complexed Waste term applied to all Hanford Site liquors not identified as complexed.. See also NCPL and NCPLEX.
NCRW	Neutralized Cladding Removal Waste—Same as CWP/Zr. See also CWP, CWP/Zr, and PW.
NDAA	National Defense Authorization Act (term located WHC-EP-0702, Rev 0)
NE	Northeast quadrant of tank (term from WHC-SD-WM-ER-204, Rev.0)
NEC	National Electrical Code (term located LA-UR-92-3196 Revised)
NEPA	National Environmental Policy Act (term located WHC-EP-0702, Rev 0)
Neutralized PUREX Acid Waste	The original plant in 1956 neutralized all of the high-level waste and sent it to the A-241 Tank Farm. As fission product recovery started, a portion of the waste was treated for Strontium Recovery and then neutralized. As of 1967 all of the High-Level Waste left PUREX as an acid solution for treatment at B Plant. See also P, and PL.
NFAW	Aging waste from PUREX/PFM high level waste.
NFPA	National Fire Protection Association (term located LA-UR-92-3196 Revised)
Neutron Probe	Probe equipped with a neutron source and detector. They are used in dry well monitoring to determine the moisture content of the soil as one way to detect leaks in underground waste storage tanks or pipelines.
nf	does not show at surface, not in a pit - no surface access

NFAW	AGING WASTE FROM PUREX/PFM HIGH LEVEL WASTE (FFTF-NCAW) See also AGE, AGING, AGING WASTE, HAW, IWW, NCAW, NHAW, NRAW, and P83-88.
NFPA	National Fire Protection Association
NHAW	AGING WASTE FROM PUREX/PFM PROCESSING OF NPR FUEL
NIOSH	National Institute of Occupational Safety and Health (term located LA-UR-92-3196 Revised)
NIST	National Institute of Standards and Technology (term located LA-UR-92-3196 Revised)
NIT	HNO_3/$KMNO_4$ solution added during evaporator operation (Neutralization in Transfer?) See also PNF.
NOx	Oxides of nitrogen (term located WHC-EP-0791)
NPH	Normal Paraffin Hydrocarbon was diluent used in Uranium recovery and PUREX processes, and is close to Dodecane, $C_{12}H_{26}$.
NRAW	AGING WASTE FROM PUREX/PFM RESIDUE ACID WASTE (FFTF-NCAW). See also AGE, AGING, AGING WASTE, HAW, IWW, NCAW, NHAW, PAW, PFM, and P83-88.
NRC	US Nuclear Regulatory Commission (term from WHC-EP-0791)
NRP82	DILUTE, NON-COMPLEXED WASTE FROM FY82 100-N AREA WASTE TRANSFER
NRPO4	DILUTE, PHOSPHATE WASTE FROM 100 N AREA
NRSO4	DILUTE, NON-COMPLEXED WASTE FROM 100 N AREA
NSTF	Near Surface Test Facility (NSTF) is a full-scale demonstration facility designed for testing, engineering, and training.
NTA	Nitrilotriacetic acid
OFFGAS	Cell air and offgas (term located WHC-EP-0791)
OP	Observation Port (term from WHC-SD-WM-ER-204, Rev.0)
Open Hole Salt Well	A well in which a pump is inserted in solid waste. Frequently used to remove the liquid from tanks containing less than 2 feet of sludge. See also Salt Well.
ORR	Operational Readiness Review (term located WHC-EP-0702, Rev 0)
OSD	Operational Safety Document
OSHA	Occupational Safety and Health Administration
OSR	Operational Safety Requirement
OTHHI	Other upper limit (term located WHC-EP-0791)
Out-of-Service	A tank which does not meet the definition of an in-service tank. All single-shell tanks are out of service.
OUTX	Transfer from Tank_n out to either a secondary processing operation or to a crib. See also TR.
OVM	Organic Vapor Monitor (term located WHC-EP-0702, Rev 0)
OWW **OWW1, OWW2, OWW3**	ORGANIC WASH WASTE FROM PUREX. Evidently, this was combined with P waste in 1960-61, but usually kept separate. The solvent used in PUREX was treated before reuse by washing with potassium permanganate and sodium carbonate, followed by dilute nitric acid and then a sodium carbonate wash. See also A-Plant, CWP, CARB, OWW PUREX Plant, and.
P	PUREX HLW, 1956-72. Sometimes assumed to be 50% OWW. Used NPH/TBP to extract both Pu and U. Np was also extracted from 1963-72. See also DN, and PL.
P	Photo Evaluation (term located Tank and Surveillance and Waste Status Summary Report)
P1	PUREX high-level waste generated between 1955 and 1962.
P2	PUREX high-level waste generated between 1963 and 1967.
P83-88	now called PXNAW or NCAW. AZ-101 and AZ-103. See also AGE, AGING, AGING WASTE, HAW, IWW, NCAW, NFAW, NHAW, NRAW, PAW, and PFM.

PL83-88	now called PXMSC
P-10 Pump	A turbine pump used in the first stage of removing liquids from a waste storage tank.
P&IDs	Piping & Instrument Diagrams
P00-P##	In-Plant scavenging with FeCN. See also SCAV, T00-T##
PADFG	PUREX AMMONIA DESTRUCTION WASTE, FROM FUELS GRADE FUEL
PADWG	PUREX AMMONIA DESTRUCTION WASTE, FROM WEAPONS GRADE FUEL
Partially Interim Isolated	The administrative designation reflecting the Interim Isolated completion of the physical effort required for Interim Isolation except for isolation of risers and piping that is required for jet pumping or for other methods of stabilization.
PAL	222-S Process and Analytical Laboratory
PAS	PUREX Acidified Sludge—refers to sludge that has been sluiced from waste tanks and acidified to 0.1 M HNO_3 (as part of Cs/Sr recovery) in AR-Vault.
PASF	PUREX AMMONIA SCRUBBER FEED. Waste that derives from the scrubber for the cladding dissolves off gas.
PASF83-88	PUREX Ammonia Scrubber Fee, never before seen
PAW	PUREX Acidified Waste. Also used to refer to Aluminum Cladded Fuel (as opposed to ZAW for Zirconium Cladded Fuel). See also AGE, AGING, AGING WASTE, HAW, IWW, NCAW, NFAW, NHAW, NRAW, PFM, and P83-88.
PCOND	PUREX condensate
PCONDCRIB	PUREX condensate to crib.
PD	PUREX decladding waste. See also CWP/Zr, NCRW, and PN.
PDBNG	DECLADDING SLUDGE (NON-TRU) FROM B PLANT PROCESSING
PDBSU	DILUTE, NON-COMPLEXED WASTE FROM B PLANT DECLADDING WASTE
PDBTG	B PLANT AGING WASTE SOLIDS FROM PUREX DECLADDING WASTE
PDCSS	DILUTE NON-COMPLEXED PUREX DECLADDING WASTE, FY 1986 ONLY
PDL87	PUREX DECLADDING SUPERNATANT, 1987
PDL89	PUREX DECLADDING SUPERNATANT, NON TRU, SPENT METATHESIS REMOVED
PD/PN	Plutonium-Uranium Extraction (PUREX) Neutralized Cladding Removal Waste (NCRW), transuranic waste (TRU). See also PUREX Decladding.
PDNSG	NON-TRU DECLADDING SLUDGE FROM PUREX
PDS87	PUREX DECLADDING SLUDGE
PDS89	PUREX DECLADDING SLUDGE AFTER FY89
PDSLG	PUREX DECLADDING SLUDGE SOL PUREX
PDSUP	DILUTE, NON-COMPLEXED WASTE PUREX DECLADDING WASTE
PFD	Process Flow Diagram (term located WHC-EP-0791)
PFeCN	Ferrocyanide sludge produced by in-plant scavenging of waste from uranium recovery.
PFeCN1	Ferrocyanide sludge produced by in-plant scavenging of waste from Uranium recovery. Used 0.005 M Ferrocyanide. See also FeCN, TFeCN, UR, P00, and T00.
PFeCN2	Same as PFeCN1, except used 0.0025 M Ferrocyanide used.
PEL	Permissible Exposure Limit
PFM	Process Facility Modification (PFM) Project provides a head end facility for the PUREX Plant in which N-fuel and FFTF fuel can be processed. See also AGE, AGING, AGING WASTE, HAW, IWW, NCAW, NFAW, NHAW, NRAW, PAW, and P83-88.
PFMMS	DILUTE, NON-COMPLEXED WASTE FROM SHEAR/LEACH PROCESSING OF NPR FUEL
PFP	Z Plant Plutonium Finishing Plant. Pu Finishing Plant waste. See also DN, DN/PD, DN/PT, P, PRF, PFPNT, PFP TRU Solids, TRU, Z Plant, and 224

PFPGR	DILUTE, NON-COMPLEXED WASTE FROM RETRIEVED PFP SOLIDS
PFPNT	NON-TRU SLUDGE FROM THE PFP SOL Z PLANT. See also DN, DN/PD, DN/PT, P, PRF, PFP TRU Solids, TRU, Z Plant, and 224
PFPPT	DILUTE, NON-COMPLEXED WASTE FROM THE PFP (WITH TRUEX). See also TRUEX
PFPSL	HIGH-TRU SLUDGE FROM THE PFP SOL Z PLANT. See also DN, DN/PD, DN/PT, P, PRF, PFPNT, PFP TRU Solids, TRU, Z Plant, and 224
PFP TRU Solids	TRANSURANIC SOLIDS FRACTION FROM PLUTONIUM FINISHING PLANT OPERATIONS. See also DN, DN/PD, DN/PT, P, PRF, PFPNT, PFP, TRU, Z Plant, and 224
PhW	Phosphorous Waste
PI	Partially Interim Isolated. The administrative designation reflecting the completion of the physical effort required for Interim Isolation except for isolation of riser and piping that is required for jet pumping or for other methods of stabilization. (term located Tank and Surveillance and Waste Status Summary Report)
PL	PUREX low-level waste. See also DN, DN/PD, DN/PT P, PL, PFP, PFP TRU Solids, PRF, TRU, PFP TRU Solids, Z Plant, and 224.
PML89	PUREX SPENT METATHESIS LIQUID AFTER FY89
PMS89	PUREX SPENT METATHESIS SOLIDS AFTER FY89
PMW	PUREX miscellaneous waste
PN	PUREX, neutralized cladding waste. See also CWP, NCRW and PD.
PNF	Partial Neutralization Feed. Indicates addition of nitric acid at an evaporator in an attempt to produce more salt cake during volume reduction. See also NIT.
PNL	Pacific Northwest Laboratory
PNW	Partial Neutralization Waste
Pond (Swamp)	Ground area where uncontaminated or low-level waste water is discharged to seep into the ground.
PP	pump pit (term located WHC-SD-WM-ER-204, Rev.0)
PRA	Probabilistic Risk Assessment
PRF	Plutonium Reclamation Facility—Type of waste generated in Z-Plant for "finishing wastes". Solvent based extraction process using CCl4/TBP. See also DN, DN/PD, DN/PT, P, PFP, PFP TRU Solids, Z Plant, 224, and 236-B.
PRTR	Plutonium Recycle Test Reactor
Primary Addition	An addition of waste from a specific plant or process vault. These additions come from the *Waste Status and Transaction Summary*., WHC-SD-WM-TI-614 & -615, Rev. O, DRAFT.
PRTR	Plutonium Recycle Test Reactor
PS	Primary Stabilization . The condition of an inactive waste storage tank after all liquid above the solids, other than isolated surface pockets has been removed. Isolated surface pockets of liquid are those not pumpable by conventional techniques.
PSA	Probabilistic Safety Assessment
PSICSF	Pump System installation containment seal fixture
PSL	PUREX sludge sluiced during recovery of Sr.
PSS	PUREX Sludge Supernatant.
PSSF	PUREX Sludge Supernatant Feed?
PT	Plutonium Finishing Plant (PFP) TRU Solids. TRU solids from 200W.
PT100	TRU waste from ??
PUREX	Plutonium Uranium Extraction Plant. Also called A Plant where PUREX process ran from Jan.1952-Jun. 1972, then was in standby and ran again from Nov. 1983 to 1991, and is now shutdown. See also A Plant, CWP, CARB, OWW , and P.
PWM	Pulse width modulated

PWR	Pressurized Water Reactor Core II from Shipping Port Atomic Power Station
PX86S	DILUTE, NON-COMPLEXED WASTE FROM PUREX MISC. STREAMS (NPR FUEL) FY 86
PXBAW	B PLANT AGING WASTE SUPERNATANT FROM RETRIEVED AGING WASTE
PXBSG	B PLANT AGING WASTE SOLIDS FROM RETRIEVED AGING WASTE
PXFTF	DILUTE, NON-COMPLEXED WASTE FROM PUREX MISC. STREAMS (FFTF)
PXLOW	PUREX LOW LEVEL WASTE THAT WENT TO SST
PXMET	PUREX DILUTE, NON-COMPLEXED DECLADDING: SPENT METATHESIS
PXMSC	DILUTE, NON-COMPLEXED WASTE FROM PUREX MISC. STREAMS (NPR FUEL)
PXNAW	AGING WASTE FROM PUREX HIGH LEVEL WASTE
QA	Quality Assurance
QATF	Quality Assurance Task Force
Questionable Integrity	Any tank that has a small decrease in liquid level or a radiation increase in an associated dry well, for which the remaining data for the tank is insufficient to support a conclusion with 95% confidence that the tank is sound.
R	REDOX High Level Waste (HLW) was generated from 1952 to 1966. It used methylisobutylketone (hexone) as a solvent, and extracted both uranium and plutonium. (S-Plant) Ran from Jan. 1952 to Dec. 1967.
R1	REDOX waste generated between 1952 and 1957.
R2	REDOX waste generated between 1958 and 1966.
R202S	
RCC	??REDOX CC??
RCOND	REDOX Condensate.
RCONDCRIB	REDOX Condensate to Crib.
REC	Receive from Trans_tank and are always positive. Trans_tank will always be one of the primary 177 waste tanks. See also SEND, TR, and XFER.
REDOX	Also know as S-Plant where REDOX process ran 1952-66? See also R, and CWR.
Removed from Service (Tanks)	Any tank that is a confirmed leaker or is not intended for reuse.
RESD	Residual Evaporator Liquor
RISER	Pipe leading into tank dome See also Blank Space.(term located SD-RE-TI-053 Rev. 8)
Riser P/CP	Riser is recessed below a cement pad with an access plate at grade (term located SD-RE-TI-053 Rev. 8)
RIX	REDOX Ion Exchange. See also RTX, and SIX
RP	Receiving Pit (term located WHC-SD-WM-ER-204, Rev.0)
RMA	Remote Mechanical A-Line.
RMC	Remote Mechanical C-Line—Process used in Z Plant.
RSltCk	Salt Cake precipitate from self-concentration in S and SX Farms.
RSN	REDOX Supernatant
RSS	REDOX Sludge Supernatant
RSS	Remote Supervisory Station
RTD	Resistance Temperature Detector (term located WHC-SD-WM-TI-553, Rev 0)
RTX	REDOX Ion Exchange. See also SIX, and RIX
S	Transaction Flag Key-Partial Neutralization (PNF).
S	Sludge Level Measurement Device (term located Tank and Surveillance and Waste Status Summary Report)
S1SltCk	Salt cake waste generated from the 242-S Evaporator/crystallizer from 1973 until 1976.

S2SltSlry	Salt cake waste generated from the 242-S Evaporator/crystallizer from 1977 until 1980.
SA	Safety Assessment
Salt Cake	Crystallized Nitrate and other salts deposited in waste tanks, usually after active measures are taken to remove moisture. (term located Tank and Surveillance and Waste Status Summary Report)
Salt Slurries	Same as DSS, estimated from chemical model by precipitation (via evaporator). DSS derives from the supernatants of a variety of wastes following evaporation of water. See also DSS, and A2Altslr.
Salt Well	A hole drilled or sluiced into a salt cake and lined with a cylindrical screen to permit drainage and jet pumping of interstitial liquors.
Salt Well Liquid	See also SWLIQ
Salt-Well Pump	A low-capacity pump used to remove interstitial liquid from wells.
SAR	*Safety Analysis Report*
SCAV	Scavenging campaign with FeCN on TBP, 1952-57. See also T00-T##, P00-P##, and Scavenged.
Scavenged	Waste which has been treated with ferrocyanide to remove cesium for the supernatant by precipitating it into the sludge. See also SCAV
SCBA	Self-contained Breathing Apparatus
SCO	*Safety Condition for Operation*
SCWO	Supercritical Water Oxidation (SCWO) destroys organics completed with metal ions and precipitates the multivalent metals out of solution as their hydroxides. Process conditions for SCWO are 500_ C and 3,000 psi. (term located WHC-EP-0791)
SD	Slurry distributor (term located WHC-SD-WM-ER-204, Rev.0)
SDRCSF	Slurry distributor removal containment seal fixture
SVOA	Semi-volatile organic analysis
SEND	Transfer from Tank_n to Trans_tank and is always negative. Trans_tank will always be one of the primary 177 waste tanks. See also TR and XFER.
SET	Connect cascaded tanks together. See also CAS and END.
SF	Slurry feed?
Side referenced tank	A dished-bottom tank where the zero point for the liquid-level gauges is at the elevation that the dished bottom begins.
SIX	REDOX Ion Exchange. See also RTX, and RIX.
SL	DOUBLE-SHELL SLURRY
SL	Sludge (Solids formed during sodium hydroxide additions to waste. Sludge usually was in the form of suspended solids when the waste was originally received in the tank from the waste generator. In-tank photographs may be used to estimate the volume.
SLS	solid/liquid separation (term located WHC-EP-0791)
SLT	sludge level tape (term located WHC-SD-WM-ER-204, Rev.0)
SL3SY	DOUBLE-SHELL SLURRY FROM EOFY 80 SY-103 INVENTORY
Sludge	Solids formed after waste neutralization with sodium hydroxide additions. Sludges usually sediment and remain in the tanks into which the waste is originally added.
SLUD31	Sludge Wash C HLW stream (term located WHC-EP-0791)
Slugs	An term for uranium fuel elements which had been machined or extruded into short cylinders which were then clad or encased in corrosion-resistant metals.
Sluicing, or Sluiced	At Hanford, this means to dissolve or suspend in solution by action of a high pressure water stream.
SLULLW	Sludge Wash C LLW stream
SMM	*Supernatant Mixing Model* that calculates the composition of tank liquids and concentrates as linear combinations of HDW supernatants.

SMP	Sludge Measurement Port (term located WHC-SD-WM-ER-204, Rev.0 & SD-RE-TI-053 Rev. 8)
SN	Sluicing nozzle (term located WHC-SD-WM-ER-204, Rev.0)
SOE	Safe Operating Envelope
SOLEX	Solvent Extraction Option (term located WHC-EP-0791)
Sound or Sound Tank	The integrity classification of a waste storage tank for which surveillance data indicate no loss of liquid from a breach of integrity.
SP	Sluice pit (term located WHC-SD-WM-ER-204, Rev.0)
SPARE	Spare riser with no current function or planned use - possible concrete plug underneath plate (term located SD-RE-TI-053 Rev. 8)
S PLANT	The facility at Hanford which contains the original extraction process for recovery of both plutonium and uranium. See also REDOX
SREX	Strontium extraction and solvent extraction.(term located WHC-EP-0791)
SPRG	Sparge-transfer of water or volume?
SR	SST SOLIDS RETRIEVED
SR	Sluicing Riser (term located WHC-SD-WM-ER-204, Rev.0)
SRCVR	Slurry Receiver Tank
SREX	Strontium extraction
SRR	Slurred PUREX sludge from A and AX Farms was sent to B Plant for strontium recovery from 1967-76. Some 801 kgal was sent to and 2,810 kgal returned from B Plant with A-102, A-106, and AX-103 as a staging tanks sending sludge to AR vault and supernatant to C-105.
SRS	Strontium Recovery Supernatant. The sludges sluiced for SRR were washed in AR vault with supernatant from C-105. The resulting supernatants were sent to CSR.
SRS	Strontium sludge
SRS	Savannah River Site (term located WHC-EP-0791)
S.S.	Evidently refers to a direct addition from plant to a cascade series that bypassed the first tank in the cascade series.
SST	single-shell tank (term located WHC-SD-WM-ER-204, Rev.0)
SSW	Strontium Semiworks. Called C Plant or Hot Semiworks earlier, was pilot for both REDOX and PUREX, Jul. 1952 to Jul. 1956. Then reconfigured for Strontium recovery pilot plant from July 1960 to July 1967. See also C Plant and HS.
STAB	Tank stabilized by removal of liquid. Both floating suction and salt-well jet pumps are used to remove liquid.
Stabilization	The removal or immobilization, as completely as possible, or the liquid contained in a radioactive waste storage tank by salt well pumping, open hole salt well pumping, adding diatomaceous earth, etc.
STAT	Tank level measurement for each quarter in kgal (1 kgal = 1,000 gallons) as reported by Anderson.
Static Tank	A tank with no significant change in liquid level or involvement in transfer operations during a stated period of time.
SU	Supernatant (Drainable Liquid Remaining minus Drainable Interstitial). Supernate is usually derived by subtracting the solids level measurement from the liquid level measurement.
SW	SST WASHED SOLIDS
SWA	Sludge Wash A (term located WHC-EP-0791)
SWB	Sludge Wash B (term located WHC-EP-0791)
SWC	Sludge Wash C (term located WHC-EP-0791)
SWLIQ	DILUTE, NON-COMPLEXED WASTE FROM EAST AREA SINGLE-SHELL TANKS
SWLQW	DILUTE, NON-COMPLEXED WASTE FROM WEST AREA SSTs

SWP	Salt well pump (term located WHC-SD-WM-ER-204, Rev.0)
SW RCR	Salt well receiver
SWPS	Salt well pump and screen (term located WHC-SD-WM-ER-204, Rev.0)
SWS	Salt well screen (term located WHC-SD-WM-ER-204, Rev.0)
T1SltCk	Salt cake waste generated from the 242-T Evaporator -crystallizer from 1951 until 1955
T2SltCk	Salt cake waste generated from the 242-T Evaporator -crystallizer from 1955 until 1965
Tank Farm	An area containing a number of storage tanks; i.e., a chemical tank farm for storage of chemicals used in a plant, or underground waste tank storage or radioactive waste.
TBP	Tri-Butyl Phosphate-waste from solvent based uranium recovery operation in '50's. Renamed to UR waste in the Defined Waste report. More usually refers to the chemical tributyl phosphate, $OP(OC_4H_9)_3$, which was used in uranium recovery and in PUREX.
TBX	Instrument leads of several kinds - usually on annulus of tank (term located SD-RE-TI-053 Rev. 8)
TC	Thermocouple (term located WHC-SD-WM-TI-553, Rev 0)
TCIX	Technetium ion exchange (term located WHC-EP-0791)
TCO	DILUTE NON-COMPLEXED WASTE FROM WEST AREA SINGLE-SHELL TANKS
TCT	Thermocouple tree
TEDF	Treated Effluent Disposal Facility
TEMP	Temperature probe (term located SD-RE-TI-053 Rev. 8)
Terminal Liquor	The liquid product from the Evaporation-Crystallization Process which, upon further concentration, forms an unacceptable solid for storage in single-shell tanks. Terminal liquor is characterized by caustic concentration of approximately 5.5 M (the caustic molarity will be lower if the Aluminum Salt Saturation is reached first). See also HDRL.
TFeCN	Ferrocyanide sludge produced by in-tank or in-farm scavenging. See also FeCN, PFeCN, UR, P00, T00.
TFEPTU	Tank Farms and Evaporator Process Technology Unit (term located SD-WM-PE-029 Rev. 0, 242-A Evap/Crystallizer FY 84-86 Campaign Run)
TGA	Thermal Gravimetric Analysis
TH	Thoria HLW or Cladding waste
TH66	
TH77	
Thermocouple Tree	A group of thermocouples assembled in a pipe and inserted into a waste tank for measuring temperatures at regular (normally 2 foot) vertical intervals.
Thermowell	A well in a waste tank which contains thermocouples
THFTCA	Tetrahydrofurantetracarboxylic acid (term located WHC-EP-0791)
THL	Thoria Low Level
TK	Tank
TK	TK-17-2 was an early name for B Plant. See also B Plant and 222-B.
TL	Terminal Liquor
TLM	*Tank Layer Model* derived from the Waste Status and Transaction Record Summary (WSTRS) database.
TLV	Threshold limit value
TLV-C	Threshold limit value-ceiling
TLV-STEL	Threshold limit value-short-term exposure limit
TLV-TWA	Threshold limit value-time weighted average
TMACS	Tank monitor and control system (term located WHC-SD-WM-TI-553, Rev 0)

TOC	Total organic carbon (term located WHC-EP-0791)
T00-##	In-Tank scavenging with FeCN. See also SCAV, P##
TP	Temperature probe (term located WHC-SD-WM-ER-204, Rev.0)
TP	Throughput nominal plant throughput PFR (Pu Nitrate), RMA (Pu Oxide), RMC (Pu Metal). See SD-WM-PE-029 Rev.0, 242-A Evap/Crystallizer FY 84-86 Campaign Run
TPA	Tri-Party Agreement includes DOE, Washington State Dept. of Ecology, and the EPA
TPLAL	DILUTE, NON-COMPLEXED WASTE FROM T PLANT
TPLAN	DILUTE, NON-COMPLEXED WASTE FROM T PLANT
T Plant	Decontamination plant for various equipment. Originally built for $BiPO_4$ process, but since only used for decontamination. $BiPO_4$ ran from Dec. 1944 to Aug. 1956. See also 222-T
TPLAS	SLUDGE FROM T PLANT OPERATIONS
TR	Transfer from tank. See also REC, SEND, and XFER
TRAC	Hanford radionuclide Tracking program devised by Jungfleisch. Also, Transient Reactor Analysis Code developed at LANL.
Trench	A deep furrow in the ground. At Hanford, they are used for the disposal of solid waste.
trFlag	Transaction Flag Keys—used by W-TRAC—See also CDF,D,E,S,SV,1,3,6,.17,.33.
TRG	Test Review Group
TRU	Transuranic. See also DN, DN/PD, DN/PT, P, PFP, PRF, Z, and 224.
TRUEX	Transuranic Extraction. See also PFPPT.
TRUEX-C	Transuranic Extraction Option C (term located WHC-EP-0791)
TRULLW	TRUEX-C LLW stream (term located WHC-EP-0791)
TRUX31	TRUEX-C HLW stream (term located WHC-EP-0791)
TSD	Treatment, Storage or Disposal Unit
TSR	Technical Safety Requirement
TTF	Thermal Treatment Facility
TWRS	Tank Waste Remediation System
TXR Vault	Vault in TX Farm used in FeCN scavenging in TX Farm.
Type I Tank	These are the 200 series tanks found in B, C, T, and U Farm. They have an operating capacity of 55,000 gal., a 20-ft., diameter, a 6-in. dish bottom, and a 3-ft. knuckle. Generation is not associated with Type I tanks.
Type II Tank	These are the original (1st generation) tank designs, which are found in B,C,T, and U (excluding the 200 series tanks), and BX Tank Farms. See also 1st Generation Tank.
Type III Tank	These are the 2nd generation tank designs, which are found in BY, S, TX, and TY Tank Farms. See also 2nd Generation Tank.
Type IV Tank	These are 3rd, 4th, and 5th generation tank designs, which are found in SX, A, and AX Tank Farms, respectively. See also 3rd Generation Tank, 4th Generation Tank, and 5th Generation Tank.
Type V Tank	These are the first double-shell tank designs, which are found in AY, AZ, and SY Tank Farms.
U1U2	DILUTE, NON-COMPLEXED WASTE FROM U1/U2 GROUNDWATER PUMPING
UFL	Upper Flammability Limit (term located WHC-EP-0702, Rev 0)
UNC	Dilute sulfate waste . See also HEDL. (see SD-WM-PE-029 Rev.0, 242-A Evap/Crystallizer FY 84-86 Campaign Run)
UNC	UNC Nuclear Industries Inc.
UNC Fuels	
UNH Stream	See 224-UA

UNKN	UNKNOWN WASTE ORIGIN SINK
UOR	Unusual Occurrence Report
U1U2	Dilute, non-complexed waste from U1/Us ground water pumping.
U Plant	Uranium Recovery Plant from Mar. 1952 to Jan. 1958, UO_3 Plant from then until Sept. 1972. Restarted in Mar. 1984, and is now shutdown. See also 222-U, UR, and TBP.
UPS	Uninterruptible Power Supply
UR	Uranium Recovery Operation in 222-U, 1952-57. Created TBP (primary waste) and FeCN (scavenging wastes). TBP waste called UR waste in Defined Waste report. See also, TFeCN, PFeCN, P00, T00, FeCN. See also TBP.
UREX	Uranium Extraction
USNRC	US Nuclear Regulatory Commission
USBM	US Bureau of Mines (term located WHC-EP-0702, Rev 0)
USNRC	U S Nuclear Regulatory Commission
USQ	Unreviewed Safety Question (term located WHC-EP-0702, Rev 0)
UX-241	???
V & V	Validation and Verification
VAQUELLW	Varied aqueous liquids (term located WHC-EP-0791)
VCBUSTL	Varied combustible solids and liquids (term located WHC-EP-0791)
VDTT	Velocity, Density, Thermocouple tree
VM	Vapor Manifold (term located WHC-SD-WM-ER-204, Rev.0)
VOF	Volume Of Fluid
VOFFGAS	Varied Cell Air and OffGas (term located WHC-EP-0791)
VNCBUSTS	Varied Noncombustible Solids (term located WHC-EP-0791)
WASHF	OUTFLOW TO SST WASH FACILITY
Waste Tank Safety Issue	A potentially unsafe condition in the handling of waste material in underground storage tanks that requires corrective action to reduce or eliminate the unsafe condition. (term located Tank and Surveillance and Waste Status Summary Report)
Watch List Tank	An underground storage tank containing waste that requires special safety precautions because it may have a serious potential for release of high-level radioactive waste because of uncontrolled increases in temperatures or pressure. Special restrictions have been placed on these tanks by "Safety Measures for Waste Tanks at Hanford Nuclear Reservation," Section 3137 of the National Defense Authorization Act for Fiscal Year 199l, November 5, 1990, Public Law 101-501 (Also known as the Wyden Amendment) (term located Tank and Surveillance and Waste Status Summary Report)
WATER	FLUSH WATER FROM MISCELLANEOUS SOURCES. See also WTR.
WC	Weather Cover (polyurethane foam) (term located WHC-SD-WM-ER-204, Rev.0)
WESF-Plant	Construction complete in 1974. Capable of producing up to 350 capsules of cesium and 175 capsules of strontium per year. 1575 cesium capsules and 625 strontium capsules produced between 1974 and 1985. See also 225-B
WHC	Westinghouse Hanford Company
WIPP	Waste Isolation Pilot Plant (term located WHC-EP-0791)
WMIS	Waste Management Information System (term located WHC-EP-0791)
WRAP	Hanford's first major solid waste processing plant, serving to analyze and repackage containers of waste left from the Hanford defense mission and generated by cleanup activities.
WSCF	Waste Sampling and Characterization Facility
WSTRS	*Waste Status and Transaction Records Summary*
WTR	Water. See also WATER.
WVDP	West Valley Demonstration Project (term located WHC-EP-0791)

WVP	Waste volume projections
WVR	Waste volume reduction
XFER	Transfer of waste out of tank. See also REC, SEND, and TR.
XIN	Addition of primary waste from plant (always positive). This transaction also covers waste returning from secondary processing operations.
Z	Z Plant waste. 234-5Z waste/Z Plant Pu Finishing. See also DN, DN/PD, DN/PT, P, PFP, PRF, TRU, and 224.
ZAW	Zirconium Acidified Waste (PUREX waste stream from Zirconium (Zircaloy II) cladded fuel.
ZHIGH	DILUTE, NON-COMPLEXED WASTE FROM THE PFP (WITHOUT TRUEX)
ZLAB	DILUTE, NON-COMPLEXED WASTE FROM PFP LABORATORIES
ZLOW	DILUTE, NON-COMPLEXED WASTE FROM PRE-FY85 Z PLANT OPERATIONS
ZPA	Zero Period Acceleration
Z Plant	Pu finishing plant. See also DN, DN/PD, DN/PT, P, PFP, PRF, TRU, Z, and 224. Operated from 1949 to 1991, and is now in standby
ZPRFL	DILUTE, NON-COMPLEXED WASTE FROM PRF PROCESSING
ZPRFS	PFP TRU SOLIDS FROM PRF PROCESSING
ZRM	Waste abbreviation
ZRMCL	DILUTE, NON-COMPLEXED WASTE FROM PFP RMC PROCESSING
ZRMCS	PFP TRU SOLIDS FROM PFP RMC PROCESSING
1AYIN	CONCENTRATED COMPLEX WASTE FROM AY-101 INVENTORY
1AZIN	PRE 2-81 AZ-101 INVENTORY
1C	1st Cycle Decontamination-BiPO4 process. Often included cladding waste. Held 10% of FP, 1% of Pu. See also BiO_4, MW, and 2 C.
1C1	First cycle decontamination waste from the $BiPO_4$ process, 1944 to 1951.
1C2	First cycler decontamination waste from the $BiP0_4$ process, 1952 to 1956.
1C44-51	Includes CW
1C52-56	Includes CW
1CEB	1st Cycle Evaporator Bottoms
1CF	??1st Cycle Feed?? Set to WATER in TRAC.
1CFeCN	Ferrocyanide sludge produced by in-plant scavenging of 1C supernatant wastes. Used 0.005 M ferrocyanide. See also FECN, PFeCN, TFeCN.
1CS	1st Cycle Scavenging waste. TY-101 and TY-103 received 1C waste that was scavenged with FeCN before it was added to the tanks. Termed 1CFeCN.
1st Generation Tank	The original tank design encompassing Tank Farms B, C, T, U (excluding the 200 series tanks), and BX. These tanks have an operating capacity of 530,000 gal, a 75-ft. diameter, a 12-in. dish bottom, and a 4-ft knuckle. Also see Type II tanks.
2C	2nd Cycle Waste from BiO_4 process. Supernatant often cribbed, 0.1% of FP, 1% of Pu. See also BiO_4, MW, and 1C.
2C1	2nd Cycle Waste from BiO_4 process, 1944 to 1951
2C2	2nd Cycle Waste from BiO_4 process, 1952 to 1956
2AYIN	PRE 2-81 AY-102 INVENTORY
2AZIN	PRE 2-81 CONCENTRATED COMPLEX WASTE FROM AZ-102 INVENTORY
2SYIN	PRE 2-81 SY-102 INVENTORY
2nd Generation Tank	Same as original tank design (1st generation or type II) except the operating capacity was increased to 758,000 gal. Also, see Type III tanks.
202-S	Also known as S-Plant where REDOX process ran 1952-66? See also R, CWR, AND S-PLANT
204-AR	Rail Car Unloading Facility, completed in 1981, replaced 204-S as Rail Car Unloading Facility. Completed in 1981.
211-T	Chemical storage area used for nitric acid and sodium hydroxide storage, low-level radioactive sludge storage.

221-B	See also B Plant
221-T	Head End facilities (two cells) in 221-T Building are used by HEDL as a containment systems test facility to develop sodium aerosol data needed for the design of air cleaning equipment for large-scale Liquid Metal Fast Breeder Reactors. 221-T Building (Cell 4) used for interim storage of Pressurized Water Reactor Core II fuel from Shippingport Atomic Power Station. See also T-Plant.
222-B	One of the three original bismuth-phosphate processing facilities. Later converted to waste fractional plant. B Plant used for $BiPO_4$ 1944-52, then for FP recovery. See also B Plant and TK.
222-C	Initially a pilot plant for REDOX, later a pilot plant for PUREX and B Plant waste partitioning. See also C Plant.
222-T	T Plant used for $BiPO_4$ 1944-52.
222-U	One of the three original Bismuth Phosphate Processing Facilities. Later converted to a uranium recovery plant. See also U Plant.
224	LaF finishing waste. 224-U Waste. See also DN, DN/PD, DN/PT, P, PFP, PRF, TRU, and Z
224-2	Same as 224?
224-AR Vault	Originally designed for treating and transferring tank farm sludges to B Plant and for interim lag storage and transfer of PUREX acid wastes to Plant. Also for lag storage of neutralized high-level waste enroute from B Plant to tank farm storage. Construction completed in 1968 put in standby mode in 1978.
224-F	224-U Waste. LaF Pu Finishing Plant. Same as Z-Plant? See also LaF.
224-U	Completed in 1944 as part of U Plant complex. Never used for original purpose used as training facility from 1944 to 1950, converted to UO_3 Plant in 1951. Plant shut down in 1972. Restarted 1984. Feedlines from REDOX and U Plant canyon disconnected. See also 224-F.
224-UA	Constructed in 1957 with six calciners installed. UO_3 Plant capability sufficient to handle UNH stream from REDOX, U-Plant, and PUREX.
225-B	See also WESF Plant
231-Z	DILUTE, PHOSPHATE WASTE FROM Z-231 LABORATORIES
241-Z	Underground sump pit.
242-A	Reduced pressure evaporator in East Area designed for 30% solids. A-102 was feed 1977-1980. AW-102 was feed 1981-present.
242-B	Atmospheric evaporator used for concentrating wastes, 1952-56. B-106 was feed tank.
242-S	Reduced pressure evaporator designed for 30% solids 1973-80. S-102 was feed '73-'77. SY-102 was feed '77-'81.
242-T	Atmospheric evaporator used to concentrate wastes. 1952-56 and 1965-76. TX-118 was feed tank.
242-Z	Waste treatment facility. Equipment was used to treat PRF waste and extract americium from the waste. Scheduled for D&D.
244-AR Vault	Originally designed for treating and transferring tank farm sludges to B Plant and for interim lag storage and transfer of PUREX acid wastes to B Plant. Also for lag storage of neutralized high-level waste enroute from B Plant to tank farm storage.
2706-T	Used as equipment low-level decontamination facility. See also T Plant, 271-T and 221-T.
271-T	Building used for chemical make-up area and dry storage, and offices. See also T Plant, 2706-T, and 221-T.
2736-ZA	Plutonium Storage and Support Facility. Used to store plutonium in a variety of forms. Plutonium packaged in metal containers. Also used for shipping, receiving, repackaging, and nondestructive analysis of plutonium. See also 2736-ZAB.

2736-ZAB	Plutonium Storage and Support Facility. Used to store plutonium in a variety of forms. Plutonium packaged in metal containers. Also used for shipping receiving, repackaging, and nondestructive analysis of plutonium. See also 2736-ZA
3AWIN	PRE 2-81 AW-103 INVENTORY
3rd Generation Tank	The first generation of the type IV tanks, contains the SX Tank Farm only. These Tanks have a 1,000,000 gal. operating capacity, a 75-ft. diameter, a 14.875-in. dish bottom, and no knuckle. See also Type IV tanks.
4th Generation Tank	The second generation of the type IV tanks, contains the A Tank Farm only. These tanks are the same as the 3rd generation except they have a flat bottom. See also Type IV Tanks.
5	B Plant Tank 5 and 6 waste.
5-6#	Cells 5&6 from B Plant
5AWIN	PRE 2-81 AW-105 INVENTORY
5th Generation Tank	The third generation of the Type IV tanks, found only in the AX Tank Farm. These tanks are the same as the 4th generation with the addition of grid drain slots beneath the steel liner bottom.

6AWIN CONCENTRATED PHOSPHATE WASTE IN AW-106 INVENTORY

Note on transactions involving:

CAS-Cascades that "overfill" are assumed to have been directed to low-level "sites" (cribs or trenches?). No MW or R was cascaded to low-level sites.

EVAP-Operations involving evaporators are assumed to change the waste by the difference in the transaction and status reports.

R-REDOX plant used concentrator 1967-72.

B-B PLANT used concentrator 1967-68.

Definitions in all caps are from the Waste Volume Projection Data Set.